

JSME 発電用原子力設備規格 設計・建設規格 事例規格 (NC-CC-002) 正誤表

～ 添付 ～

No.	ページ 番号	規格番号	誤	正	備考
添付					
1	添付-8	XX-3000 参考文献 (10)	(10)Hughes N R, Clarke W L, Delwiche D E, Intergranular Stress-Corrosion Cracking Resistance of Austenitic Stainless Steel Castings”, <i>Stainless Steel Casting, ASTM STP 756</i> (1982), pp. 26-470, ASTM	(10)Hughes N R, Clarke W L, Delwiche D E, Intergranular Stress-Corrosion Cracking Resistance of Austenitic Stainless Steel Castings”, <i>Stainless Steel Casting, ASTM STP 756</i> (1982), pp. 26- <u>47</u> , ASTM	NC-CC-002 のみ
2	添付-8	XX-3000 参考文献 (17)	(17)CASES OF ASME BOILER AND PRESSURE VESSEL CODE ,Case N-580-1 Use of Alloy 600 with Columbium Added,Section III,Division 1.	(17)CASES OF ASME BOILER AND PRESSURE VESSEL CODE ,Case N-580- <u>2</u> Use of Alloy 600 with Columbium Added,Section III,Division 1.	NC-CC-002 のみ
3	添付-8	XX-3000 参考文献 (19)	(19)T. Yonezawa, N. Sasaguri, K. Onimura, Effect of Metallurgical Factors on Stress Corrosion Cracking of Ni-base Alloys in High Temperature Water, <i>Proceeding of the 1988 JAIF International Conference on Water Chemistry in Nuclear Power Plants</i> , Vol.1(1988), pp.490-495.	(19)T. Yonezawa, N. Sasaguri, K. Onimura, Effect of Metallurgical Factors on Stress Corrosion Cracking of Ni-base Alloys in High Temperature Water, <i><u>Proceedings</u> of the 1988 JAIF International Conference on Water Chemistry in Nuclear Power Plants</i> , Vol. <u>2</u> (1988), pp.490-495.	NC-CC-002 のみ
4	添付-8	XX-3000 参考文献 (21)	(21)T. Yonezawa, K. Onimura, T. Kusakabe, N.Sasaguri, H. Nagano, K. Yamanaka, T. Minami, M. Inoue, Effect of Heat Treatment on Corrosion Resistance of Alloy 690, <i>Proceeding of the 2nd International Symposium on Environmental Degradation of Materials in Nuclear Power Systems-Water Reactors</i> , (1985),pp. 593-600.	(21)T. Yonezawa, K. Onimura, T. Kusakabe, N.Sasaguri, H. Nagano, K. Yamanaka, T. Minami, M. Inoue, Effect of Heat Treatment on Corrosion Resistance of Alloy 690, <i><u>Proceedings</u> of the 2nd International Symposium on Environmental Degradation of Materials in Nuclear Power Systems-Water Reactors</i> , (1985),pp. 593-600.	NC-CC-002 のみ

No.	ページ 番号	規格番号	誤	正	備考
5	添付-9	XX-3000 参考文献 (23)	(23)T. Yonezawa, K. Onimura, N. Sakamoto, N. Sasaguri, H. Nakata and H. Susukida, Effect of Heat Treatment on Stress Corrosion Cracking Resistance of High Nickel Alloys in High Temperature Water, <i>Proceeding of the International Symposium on Environmental Degradation of Materials in Nuclear Power Systems-Water Reactors</i> , (1983), pp.354-367.	(23)T. Yonezawa, K. Onimura, N. Sakamoto, N. Sasaguri, H. Nakata and H. Susukida, Effect of Heat Treatment on Stress Corrosion Cracking Resistance of High Nickel Alloys in High Temperature Water, <i>Proceedings of the International Symposium on Environmental Degradation of Materials in Nuclear Power Systems-Water Reactors</i> , (1983), pp. <u>345</u> -367.	NC-CC-002のみ
6	添付-9	XX-3000 参考文献 (31)	(31)R.W. Staehle and J.A. Gorman, Quantitative Assessment of Submodes of Stress Corrosion Cracking on the Secondary Side of Steam Generator Tubing in Pressurized Water Reactor: Part 1, <i>Corrosion</i> , Vol.59, No.11(2003), pp.931-994.	(31)R.W. Staehle and J.A. Gorman, Quantitative Assessment of Submodes of Stress Corrosion Cracking on the Secondary Side of Steam Generator Tubing in Pressurized Water <u>Reactors</u> : Part 1, <i>Corrosion</i> , Vol.59, No.11(2003), pp.931-994.	NC-CC-002のみ